

DOE Workshop on Nuclear Physics and Related Computational Science R&D for Advanced Fuel Cycles

Bethesda, Maryland
August 10 – 12, 2006

Nuclear Data Working Group
Integral Validation and Quality Assurance

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Integral Validation and Quality Assurance

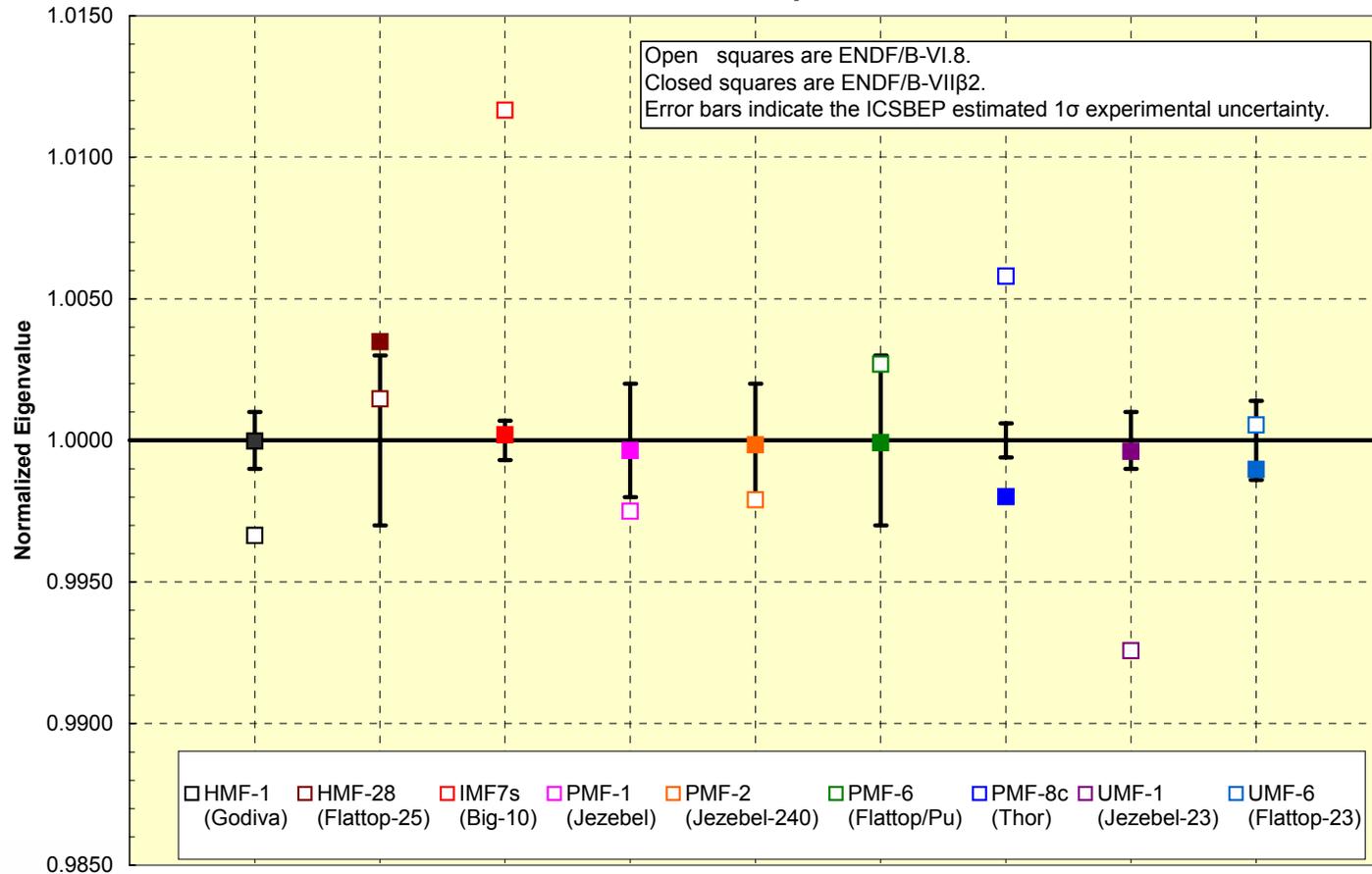
- International Handbook of Evaluated Criticality Safety Benchmark Experiments [NEA/NSC/DOC(95)03].
 - Primary data source for criticality benchmark testing of cross section data files.
 - Various fissile system categories (HEU, IEU, LEU, Pu, ^{233}U , MIXed (e.g., Pu-U) and SPECial (e.g., ^{237}Np)).
 - Various material specifications (METal, COMPOund (e.g. UO_2), SOLution).
 - Various spectral specifications (FAST, INTERmediate, THERMal).
 - New and/or revised benchmark evaluations are reviewed and approved by the International Criticality Safety Benchmark Evaluation Project (ICSBEP).
 - Updated editions of the Handbook have been published in their entirety annually since 1995.
 - The 2005 edition includes 416 evaluations that specify 3649 critical, near critical or subcritical configurations.

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- ENDF/B neutron cross section libraries have received extensive criticality testing with ICSBEP benchmarks.
 - Data testing results have been reported at regular Cross Section Evaluation Working Group (CSEWG) meetings for a variety of continuous energy Monte Carlo programs, including
 - MCNP, versions 4 and 5 (LANL and Europe);
 - RCP01 (Bechtel Bettis (Naval Reactors));
 - RACER (KAPL (Naval Reactors));
 - KENO/CENTRM/SCALE (ORNL);
 - VIM (Argonne);
 - COG, TART (LLNL);
 - Tripoli (France).

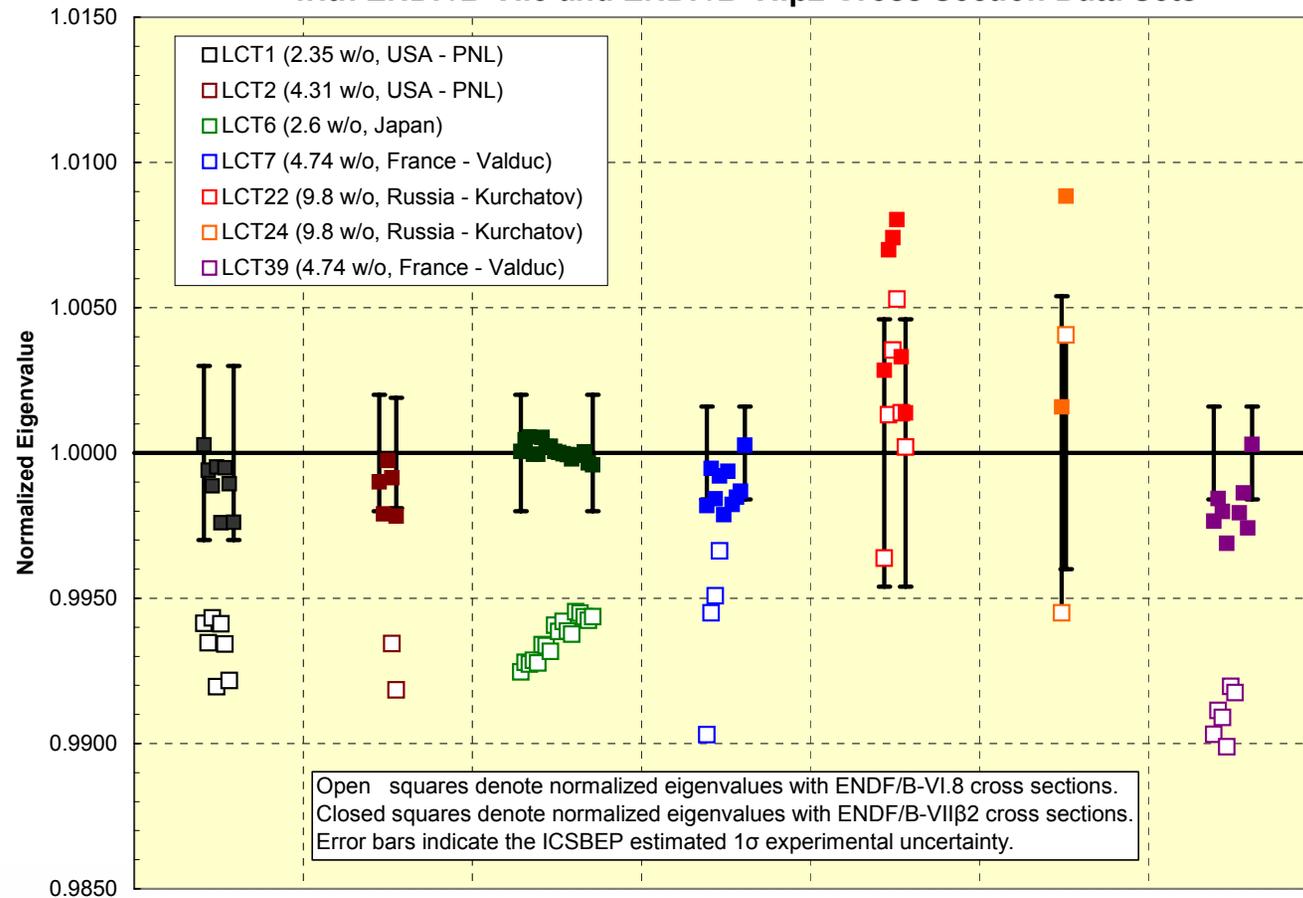
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Calculated Eigenvalues for LANL HEU, Pu and ^{233}U Unmoderated Benchmarks with ENDF/B-VI.8 and ENDF/B-VII β 2 Cross Section Data Sets



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Calculated Eigenvalues for LEU-COMP-THERM Benchmarks
with ENDF/B-VI.8 and ENDF/B-VII β 2 Cross Section Data Sets



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- Comparisons with LANL and Europe (Petten) for ENDF/B-VII β 2.
 - NJOY/MCNP used (independently) at both institutions.
 - 216 ICSBEP benchmarks in common.
 - Petten/LANL eigenvalue ratio varies from 0.9964 to 1.0036.
 - 2/3^{rds} of these ratios fall between 0.9990 and 1.0010.
- Comparisons with LANL and Europe (France) for ENDF/B-VII β 2.
 - NJOY/MCNP used at LANL; NJOY(CALENDF)/Tripoli use at Cadarache.
 - 42 ICSBEP benchmarks in common.
 - Tripoli/MCNP ratio varies from 0.9996 to 1.0010.
 - Common benchmarks include HEU-MET-FAST, IEU-MET-FAST, PU-MET-FAST, HEU-SOL-THERM and LEU-COMP-THERM.
- Comparison with LANL (MCNP) and Naval Reactors (RCP01/RACER).
 - Excellent agreement in calculated eigenvalues has been observed for many years (mostly HEU-SOL-THERM).

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- Future benchmark calculations need to include important reactor design parameters other than the critical eigenvalue
 - depletion inventories;
 - reaction rates;
 - reactivity coefficients.
- An important new source of such benchmark data will be the International Handbook of Evaluated Reactor Physics Benchmark Experiments [NEA/NSC/DOC(2006)1].
 - First published in March, 2006.
- Significant computing resources are required to perform such calculations with Monte Carlo codes.
 - Tens of thousands of CPU hours may be needed to perform a single continuous energy Monte Carlo lifetime calculation for a given design.
 - A design iteration may require dozens of lifetime calculations.

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- Conclusion
 - The excellent agreement observed from an extensive comparison of calculated eigenvalues for a wide range of ICSBEP benchmarks, using independently derived model input decks (when the codes are the same), independent codes and “independent” processing of basic cross section data files provides a high degree of assurance that today’s data processing and continuous energy Monte Carlo codes can be used to accurately calculate a wide variety of fissile systems.